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Feasibility Studies for Simultaneous Irradiation of NBSR & MITR Fuel Elements in the BR2 Reactor

S. Kalcheva, S. Van Dyck, S. Van den Berghe, G. Van den Branden

IGORR 18th: International Group on Research Reactors, Sydney, Australia, December 3-7, 2017





Feasibility Studies for Simultaneous Irradiation of NBSR & MITR Fuel Elements in the BR2 Reactor

Steven Van Dyck Manager of BR2 Reactor Sven Van den Berghe BR2 Reactor Stakeholder

Geert Van den Branden Head of Reactor Control & Experiments Silva Kalcheva Reactor Core Load Manager Nuclear Materials Science Institute SCK•CEN, BR2 Reactor, 2400 Mol – Belgium

Outline



Introduction

- Purpose of <u>Design</u> <u>Demonstration</u> <u>Element</u> tests
- Technical requirements for DDE
- MCNP calculation methodology
 - Full core 3-D MCNP modeling of BR2 reactor
 - 3-D modelling of DDE-MITR and DDE-NBSR
 - 3-D power and burn-up evolution simulation
- Calculation results

Summary

Introduction



- Four <u>Design Demonstration Element</u> (DDE) tests foreseen in the <u>US High Performance Research</u> <u>Reactor</u> (USHPRR) conversion program:
 - Missouri University Research Reactor (MURR),
 - Massachusetts Institute of Technology Reactor (MITR),
 - National Bureau of Standards Reactor (NBSR) and
 High Flux Isotope Reactor (HFIR)
- BR2 Reactor along with other MTR (ATR) involved in preliminary feasibility studies

Purpose of DDE tests



 Qualification of the new LEU fuel for each DDE in the BR2 reactor at conditions that are similar for reactor of origin



Present study: scenarios for simultaneous irradiation of MITR and NBSR Test Assemblies

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Technical requirements for DDE tests

- Simultaneous irradiation → challenge for core management
- DDE-MITR
 - $\circ Q_{max}$ (BOL)=64 W/cm²
 - F_{max} (EOL)=5.8E21 fiss/cm³

DDE-NBSR

•
$$Q_{max}$$
 (BOL)=**160** W/cm²

• F_{max} (EOL)=7.9E21 fiss/cm³





Geometry & dimensions of DDE's

DDE-MITR

 rhomboid form
 19 plates
 variable meat thickness







SCALE 1

Dimensions in mm

DDE-NBSR

- Upper and lower sections
- Divided by water gap
- Total 2 x 17=34 plates







Description of BR2 reactor characteristics

- Hyperboloid core composed of twisted and inclined reactor channels
- HEU core positioned inside and reflected by beryllium matrix
- Flexible BR2 power 40 to 100 MW
- 6-8 operation cycles per year (each 3-4 weeks long)
- High power density
 - 470 W/cm² nominal
 - 600 W/cm² admissible
- Maximum neutron flux
 - 1,2 x 10¹⁵ n/cm²/s thermal
 - 8,4 x 10¹⁴ n/cm²/s fast





Flexible BR2 reactor core loadings



- Variable core configuration variable number of CR's, FE's
- Fuel elements initial U5 burnup between 0% and 50%



MCNP6 3-D full core modeling of BR2

- Automatic burn-up & criticality simulation
- 3-D whole core geometry & depletion
- Follow-up irradiation history of each FA







- Chosen channel H5 (D=200 mm)
- Optimization of position needed for axial profile
 - Z = -15.28 cm to +43.17 cm
- Special loadings (DG's) in surrounding channels
- Choice of appropriate plug material (Al-alloy, Be)
 - 3 designs for outer plug proposed (Be, Al-alloy, Al-alloy with holes)
 - •absorbing rods (Co, W, ..) in inner/outer plug



Modeling optimizations for DDE-MITR (cont'd)



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Final design PLUGS for DDE-MITR



- Inner plug: Al-alloy (to lower the heat flux)
- Outer plug:
 - Al-alloy with 12 holes filled with rods (Co, W, Al, etc.)





Calculation model of DDE-MITR

- MCNP model of the DDE-MITR-FE contain 19 plates
- Fuel meat thickness varies from 0.33 mm to 0.635 mm
- Each fuel plate modeled with uniform mesh
 - 4 in the transverse direction (13.2 mm wide)
 - 18 in the axial direction (31.6 mm long)
 - Total number of fuel zones: 19x4x18=**1368**







Summary of DDE-MITR-FE parameters

Parameter	value
Axial position in H5 channel relatively to reactor core mid-plane, mm	+123.8
Outer/inner radius of outer fresh Be-plug, mm	99.8/75.0
Radius of inner Al-plug, mm	72.50
U _{total} density, g/cm ³	15.3
U10Mo density, g/cm ³	17.0
²³⁵ U enrichment, %	19.75
²³⁶ U enrichment, %	0.24
Meat thickness of fuel plates 1 & 19, mm	0.33
Meat thickness of fuel plates 2, 3,17 & 18, mm	0.432
Meat thickness of fuel plates 4 to 16, mm	0.635
Plate thickness, mm	1.245
2 exterior water channels thickness, mm	1.618
Plate length, mm	584.200
Meat length, mm	568.325
Diameter of absorbing rods in outer/inner plug, mm	10/20

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Modeling optimizations for DDE-NBSR

- Chosen channel H3 (D=200 mm)
- Inner plug (Al-alloy)
- Outer plug (Be)
- Optimization of water gap between upper and lower plate
 - water gap=15.24 cm (original design)
 - water gap= 5.24 cm & shift NBSR-FE UP by ∆Z=+5 cm (preferred scenario)
 - water gap = 5.24 cm & shift NBSR-FE DOWN ΔZ =-5 cm



Calculation model of DDE-NBSR

- All fuel plate dimensions not changed (original)
- 17 fuel plates in upper position
- 17 fuel plates in lower position
- 3 azimuth zones in each plate (each ~20 mm long)
- 14 axial zones (each ~20 mm long)
- Total number of fuel zones
 2x17x3x14=1428









Summary of DDE-NBSR-FE parameters

Parameter	value
Axial position in H3 channel relatively to reactor core mid-plane, mm	+100.0
Outer/inner radius of outer fresh Be-plug, mm	99.8/75.0
Radius of inner 60%Al+40%Be-plug, mm	72.50
U _{total} density, g/cm ³	15.3
U10Mo density, g/cm ³	17.0
²³⁵ U enrichment, %	19.75
²³⁶ U enrichment, %	0.24
Meat thickness of all (17 lower + 17 upper) fuel plates, mm	0.22
Plate thickness, mm	1.27
36 water channels thickness, mm	2.95
2 dummy exterior plates thickness, mm	1.65
Meat length (upper, lower plate), mm	279.4
Plate length (upper, lower plate), mm	330.2
Water gap between lower and upper plates, mm	52.4



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DDE-MITR: Heat flux at BOL (Al rods inner plug)

Be (outer plug), Al-alloy (inner plug)
 Q_{max} =158 W/cm²; Q_{min} =31 W/cm²



DDE-MITR: Heat flux at BOL (Co, W rods inner plug)



Be (outer plug), Al-alloy (inner plug)
 Q_{max} =65 W/cm²; Q_{min} =15 W/cm²



DDE-MITR: Heat flux at BOL (Co, W rods outer plug)



- Proposed geometry design: Al-alloy (inner & outer plug)
- $Q_{max} = 67 \text{ W/cm}^2$; $Q_{min} = 16 \text{ W/cm}^2$





DDE-MITR: Burn-up at 250 F.P.D. ~ BR2 10 cycles

Axial distributions at 250 days for proposed design

Maximum U5 burn-up: 55%

Maximum fission density: 3,5E21 fiss/cm³





DDE-MITR: Burn-up at 630 F.P.D. ~ BR2 26 cycles

Axial distributions at EOL for proposed design

- Maximum U5 burn-up: 90%
- Maximum fission density: 5,8E21 fiss/cm³



DDE-MITR: Burn-up evolution (proposed design)

- Time evolution of U5 burn-up (left) and fission density (right) during ~ 26 BR2 operation cycles:
 - scenario with cobalt rods inner/outer plug
 - for Q_{max} =67 W/cm² at BOL



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DDE-NBSR: Heat flux at BOL



• BOC #1

- Q_{max} =180 W/cm²; Q_{min} =155 W/cm² (lower plate)
- $Q_{max} = 145 \text{ W/cm}^2$; $Q_{min} = 40 \text{ W/cm}^2$ (upper plate)



.8×1051

7x102

6×102

5×1051

r ³rd ^{azim}uth region

DDE-NBSR: Burn-up at 350 F.P.D. ~ BR2 15 cycles

Axial distributions for proposed design

- ✤Maximum U5 burn-up: > 90%
- Maximum fission density: 7,2E21 fiss/cm³



. 9×1051

8×1021

6×1021

5×102

×1021

Sict at inuity region

2nd atimuth region

DDE-NBSR: Burn-up at 600 F.P.D. ~ BR2 25 cycles

Axial distributions at EOL for proposed design

- Maximum U5 burn-up: 100%
- Maximum fission density: 8,4E21 fiss/cm³





DDE-NBSR: Burn-up evolution during 25 cycles

Time evolution of U5 burn-up (left) and fission density (right)



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Summary

• DDE-NBSR

- Location in one of the 200 mm diameter channels (H3, H4 or H5)
- Reduced water gap between upper and lower fuel plates
- Use of pure Be-outer plug and Al-alloy for inner plug
- Negligible reactivity effect of DDE-NBSR vs. standard Be-plug
- Target performances (BOL) are met: Q_{max}=170 W/cm²
- Target performances (EOL)
 - * ~12-15 cycles > 90% U5 burn-up, 7,2E21 fiss/cm³
 - ☆ ~15-25 cycles saturation of U5 burn-up, 8,4E21 fiss/cm³

⇒ 12 cycles sufficient for maximum fission density ≤ 7E21 fiss/cm³

Summary (cont'd)



• DDE-MITR

- Location in one of the 200 mm diameter channels (H3, H4 or H5)
- Negligible reactivity effect of DDE-MITR vs. standard Be-plug
- Loading of absorbing devices in surrounding channels
- Inner plug Al-alloy
- Outer plug: Be or Al-alloy with holes for absorber rods (Co, W)
- Target performances (BOL) are met: Q_{max}=65 W/cm²
- Target performances (EOL) are met: F_{max}=5.8E21 fiss/cm³
 - 25-26 BR2 cycles for Q_{max}(BOC1)=65 W/cm²
 - < 20 BR2 cycles if maintain Q_{max}(BOCi)=65 W/cm² in cycles 'i'
 - < 20 BR2 cycles if increase target heat flux</p>

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Registered Office: Avenue Herrmann-Debrouxlaan 40 – BE-1160 BRUSSELS Operational Office: Boeretang 200 – BE-2400 MOL

