

# Safety Reassessment of German Research Reactors after the Accident at the Fukushima Daiichi NPP – Status of Improvements focused on Emergency Preparedness

Workshop on Safety Reassessment of Research Reactors in the Light of the Lessons Learned from the Fukushima Daiichi Accident

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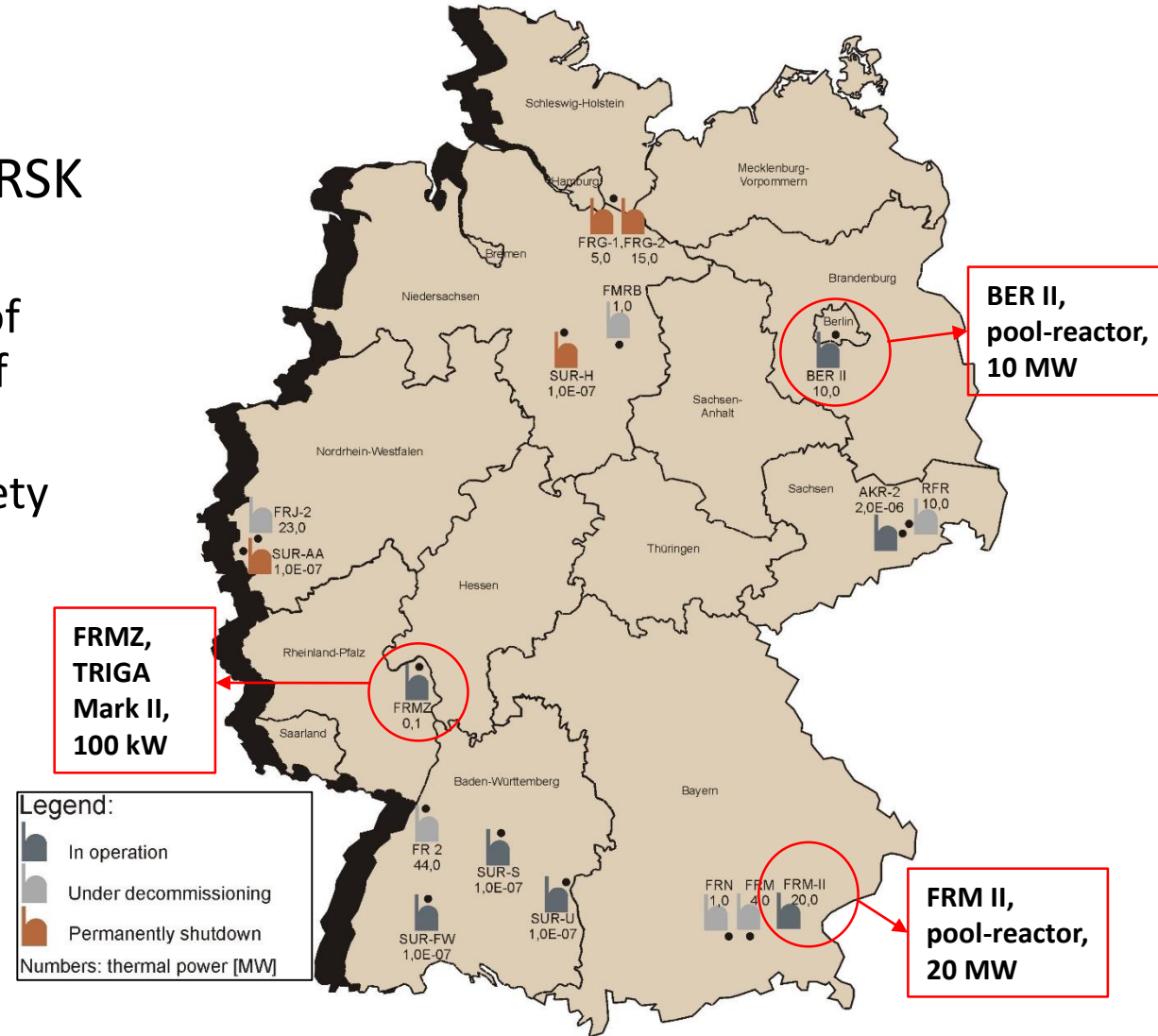
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# Safety Assessment of German RR after the Accident in Fukushima - Overview

- 07.07.2011 – BMU (Federal Ministry for Environment, Nature Conservation and Nuclear Safety) requested the RSK (Reactor Safety Commission) for a safety assessment of all research reactors with a continuous thermal power  $P_{th} > 50$  kW
- 03.05.2012 – First statement of the RSK on the robustness of German RR published
- 14.01.2015 – Safety reassessment of research reactors by the RSK on ministerial behalf began – status of implementation of improvement actions identified in 2012
- 22.03.2017 – Second statement of the RSK on the robustness of German RR published

# Facility Specific Safety Approach

- Catalogue of requirements developed by the RSK based on the stress test for NPPs
  - Adjustment of assessments criteria in a meaning of graded approach, accounting for a risk potential of individual research reactors
  - Verification of compliance of the fundamental safety functions
  - Identification of safety margins
  - Robustness of instrumentation for monitoring of reactor and radiological parameters



# Assessment Criteria

- Natural hazards
  - Earthquake
  - Flooding
  - Extreme weather conditions
- Events combinations and expanded postulated events
- Precautionary measures
- Emergency preparedness
- Man-made hazards
  - Blast wave
  - Explosive materials
  - Toxic gases
  - Terrorist hazards / aircraft crash

3 Robustness Levels ↑

3 Degrees of Protection ↑

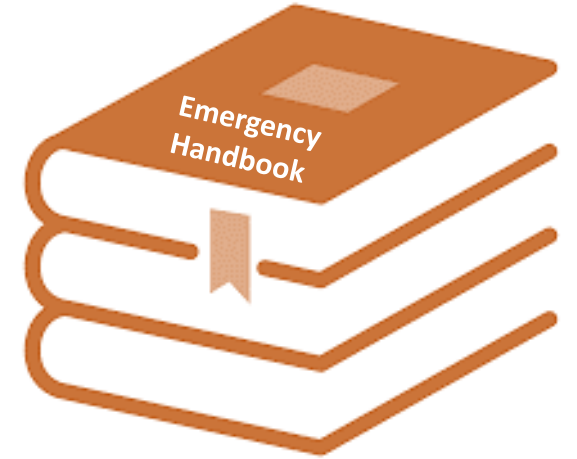


# Safety Assessment of German RR – Outcome

- Robustness of RR confirmed, but safety margins differs depending on the facility and assessment criterion
- Major potential for improvements: preventive and mitigative emergency control measures
- Need for (further) development of plant-specific measures independently from the external disaster measures, e.g.:
  - Emergency preparedness part of operating regulations
  - Establishment of emergency response team
  - Consideration of aggravated boundary conditions (e.g. damaged infrastructure and communications equipment, increased dose rate, hydrogen generation)
  - Failure of the monitoring instrumentation
  - Loss of power supply
  - Loss of coolant → supply alternatives and/or sealing of the reactor tank
  - Limitation of activity release in case of core meltdown

# Emergency manual and response team

- Preventive and mitigative emergency control measures elaborated in emergency manual
- Concept consistent with the guidance for nuclear power plants, measures adequate to the risk potential of the individual research reactors emergency manual



- Emergency response team
  - Primary reactor staff
  - Management of research centre/university may be involved
  - Consideration of external organisations

Description of emergency response team easily understandable in any case?

# Emergency water supply and sealing of reactor pool

## Redundant, divers and physical separated systems for emergency water supply

### FRM II and BER II

- Multiple water feeding: system for back-feeding for “normal” operational leakage, pipe connection, mobile fire pumps
- Various water reservoirs available, e.g.: drinking water system, storage tank, local lake / stream
- Sealing of reactor pool – individual actions depending on the leakage position
- Large leakage in the reactor pool – replacement of the fuel element into the set-down pool

### FRMZ

- Recriticality of the core due to loss of coolant – practically excluded
- Divers cooling – air
- Emergency core cooling – mobile fire pumps

Measures for water supply without entering the reactor hall?



# Emergency power supply

Redundant, diverse and physical separated systems, despite of a very limited demand

## FRM II

- Operational demand for power supply – first 3 hours after the reactor shutdown
- Emergency power supply systems: distribution grid, diesel generators, power system supplied from a third grid/mobile emergency diesel generators, emergency transformer on the site

## BER II

- Operational demand for power supply – first 10 minutes after the reactor shutdown
- Emergency power supply systems: emergency diesel generators, batteries, redundant connection points for mobile emergency diesel generators

## FRMZ

- Maintenance of fundamental safety functions requires no power supply
- Emergency power supply only for instrumentation for monitoring of the reactor and radiological parameters – diesel generators foreseen

Mobile diesel generators in place or rental contract with an external company?



# Mitigation of radioactivity release in case of core melt down

- Covering of the reactor core entirely under water, inkl. measures for long-lasting situations
- Confinement of radioactive material in means of, e.g. containment isolation, negative pressure in the reactor hall, exhaust air filtering and ventilation isolation system
- FRMZ reactor - core melt down practically excluded



# Aggravated boundary conditions

- FRM II and BER II – no power supply needed to maintain the fundamental safety functions
- FRMZ – neither power and water supply nor personnel needed to maintain its fundamental safety functions
- Adequate measures for emergency water and power supply
- Diverse systems for reactor shutdown, where appropriate
- Clearance of major infrastructural damages with support of technical assistance organizations



# Instrumentation and equipment for emergency preparedness



- Monitoring of reactor and radiological parameters
  - Instrumentation designed to withstand high temperatures, humidities and radiation dose rates
  - Reactor and radiological parameters measured continuously
  - Parameters displayed in both, the control room and in the emergency control room (FRM II, BER II)
  - Redundant battery supplied and mobile instrumentation equipment available

Robustness verification incl. hypothetical effects of internal and external DEC?

- Communication

- Large number of adequate communications systems available at all facilities, e.g.:
- Telephone system equipped with emergency batteries
- Dedicated lines
- Emergency alarm button to alert the police or fire department

Emergency secured priority line for the public telephone network?



# Education and training

- Regular training courses and emergency exercises including theory and practice of radiation protection and emergency preparedness
- Participation of the external organisations, e.g. fire departments, to ensure their knowledge about the facility and the site



## Suggested minimal frequency of emergency exercises:

- At least one per year for the on-site emergency preparedness, including the entire emergency response team
- At least every five years big exercises under assumption of severe scenarios and requiring participation of external disaster control organisations

# Conclusions



- Reassessment process finalized in March 2017
- Recommendations of the RSK from 1<sup>st</sup> step of the safety assessment principally implemented
- Measures of emergency preparedness strengthened accounting for severe scenarios, including long-lasting situations and aggravated boundary conditions
- Robustness of German research reactors generally at the high level, but further optimisation is possible, as always

# Any questions?



# Thank you for your attention!