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POST IRRADIATION TESTING CAPABILITIES OF EXPERIMENTAL REACTOR COMPONENTS AT THE LECI FACILITY FOR SERVICE LIFE ASSESSMENT

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Assessment of the environmental degradation of experimental core components is crucial for:

- Safe et efficient operation of experimental reactors
- Determination of achievable service time and replacement schedule

Feedback is needed in particular on:

- Mechanical properties evolution under irradiation
- Amount of AI to Si transmuted
- Corrosion behavior
- Swelling



[Monographies DEN, Research Nuclear Reactors, CEA, 2012]

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THE LECI FACILITY

- Hotlab located in Saclay, near Paris, France
- Three lines of hotcells
- Mission: characterization of non-fissile irradiated materials
 - Determination of behavior in normal, accidental or storage conditions
 - Prediction of in-service lifetime
- Materials studied:
 - Gen 2&3 reactors (RPV Steel, Zr alloys)
 - MTR (Aluminum alloys)
 - Gen 4 reactors (sodium or gas cooled FBR)







CHARACTERIZATION OF MATERIALS AT THE LECI FACILITY







MECHANICAL BEHAVIOR UNDER IRRADIATION (2/2)

Fracture toughness Testing Servo-Hydraulic Machine (100 kN)



350 J Charpy Impact Testing







Application to surveillance programs of core components for MTRs and NPPs

Compact Tension unconventional thickness

Internal conical mandrel (ICM)

Ductile-Brittle transition (impact tests)





Compact Compact Tension Tension (type CTR5) (type CTK)





Microscopic characterization





Dose: 4x10²¹ n/cm² Ductile behavior

Fractography of broken tensile sample surfaces



Dose: 11.7x10²² n/cm² Brittle behavior

[B.Kapusta, INSTN, Aluminum Behavior Under Irradiation, 2011]

Example of foiling corrosion on 5754-NET-O alloy with important hardening





MICROSTRUCTURAL EXAMINATIONS (2/2)



Sample preparation by nuclearized FIB or electropolishing





500nm



Nuclearized FIB-SEM (Zeiss Auriga 40)







Nuclearized Atom Probe



Nota : This work profited from a French government grant managed by the National Agency of Research under the program "Investments for [K. Buchanan et al. Acta Materialia, v132, 2017] the future" (ref. ANR-11-EQPX-0020

Precipitate-free zone in heavily *irradiated* 6061-T6 (2x10²³ n_{th}/cm²)



[B.Kapusta, INSTN, Aluminum Behavior Under Irradiation, 2011] Mg-Si needles in Si-enriched 6061 allov



CHEMICAL ANALYSES (1/2)



Nuclearized Electron Probe Micro-Analyzer

Examples of EPMA analyses on 5754-NET-O

Observation of precipitation and radiation-induced migration of chemical elements





 $\Phi_{\text{th}} (0,0254 \text{ eV}) = 8,2 \text{ x} 10^{22} \text{ n/cm}^2 \quad \Delta Si_{\text{wt}} = 2,1\%$

[B.Kapusta, INSTN, Aluminum Behavior Under Irradiation, 2011]

CHEMICAL ANALYSES (2/2)



Nuclearized Laser-Induced Breakdown Spectrometer .



LIBS technique: measurement of Si content on centimeter size samples

→information on received dose (transmutation AI-Si)

 Also available ICP-AES technique on Al shavings from massive components
→information on Mg, Fe, Si contents



CORRELATION SI CONTENT AND MECHANICAL BEHAVIOR (2/2)



Generated Si content (from AI transmutation) is directly linked to embrittlement and hardening of the components of MTRs



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CORRELATION SI CONTENT AND MECHANICAL BEHAVIOR (2/2)





CONCLUSIONS

- Methods and experimental capacities have been developed over the years at the LECI facility in order to characterize non-fissile materials.
- These means can be applied to MTRs core components such as pressure vessels, experimental racks, neutron guide tubes for example.
- Several aspects of the impact of irradiation on aluminum components can be studied:
 - Degradation of mechanical properties (hardening, embrittlement)
 - Microstructure evolution (radiation-induced segregations, precipitation)
 - Enhanced corrosion
 - Swelling
- Application to
 - Surveillance programs of MTRs
 - Prediction of component lifetime/replacement



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