

AMMON: An experimental program in the EOLE critical facility for the validation of the JHR Neutron and Photon HORUS3D calculation scheme

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The EOLE critical facility of the CEA/Cadarache



Critical facility dedicated to the neutronic studies of water moderated lattices, such as PWR or BWR cores

(First criticality 1965)

- Water reactor composed of a cylindrical vessel with an over structure of stainless steel able to contain various types of cores and associated structures.
- Water circuit designed to control the volume, the boron content and the temperature of the moderator.
- Four safety rods and one pilot rod, with different possible designs
- A set of grids allows the insertion of UOx and MOX pins



FLUOLE configuration : Analysis of the PWR core-reflector boundaries and vessel fluence estimations

The motivations of the AMMON experimental program

- Specificities of the JHR reactor for the qualification of the calculation scheme:
 - New type of fuel => U_3Si_2 ($\epsilon \ge 20\%$)... and future UMo/AI ($\epsilon = 20\%$)
 - New materials : Al in high proportion, Be, Hf
 - Innovative sub-assembly and control rod geometries
 - Irregular core lattice
- The current uncertainties of the calculation scheme for the JHR reactor are based on :
 - <u>Over-estimated</u> uncertainties, coming from the elementary qualification and propagation studies
 - <u>Safety margins</u> linked to the representativeness of non specific experiments
- A specific qualification is required in order to :
 - <u>Consolidate and reduce</u> these over-estimated uncertainties
 - Obtain <u>validated uncertainties</u> coming from measurements
 - Eliminate <u>the supplementary margins</u> for the representativeness aspects
- Safety approach
 - The JHR is an innovative reactor and <u>few representative measurements exist</u>
 - Requirement for demonstrating the uncertainties taken into account for the JHR design
 - The calculation schemes are based on new models
- Possibilities of qualifications in EOLE >> possible measurements in the JHR reactor
 - Well known and controlled environment, accessibility, dedicated instrumentation, flexibility



Objectives for the uncertainties of the program





| Parameters | Current uncertainties (2σ) | Objectives (2σ) | |
|-----------------------|-------------------------------|--|--|
| Reactivity | ±1.6\$ | <i>±0.6</i> \$ | |
| Power peak | <u>±</u> 8% | De ±2 à ± 5% | |
| Flux | ±20% | De ±5 à ± 10% | |
| Absorbers weight | ±8% | ±5% | |
| Exp. devices weight | ±10% | Depends on the device | |
| Gamma Heating | ±30% | <±20% | |
| Moderator coefficient | 5 ^{E-3} \$∕°C | 2.5 ^{E-3} to 4 ^{E-3} \$/°C | |
| Kinetics parameters | Non evaluated | ±10% | |

The AMMON design in EOLE



The AMMON alternative configurations







The basis of the AMMON design





Measurements and associated uncertainties (1/4)



| | Parameters | Type of measurements | Uncertainty (2σ) | |
|--------|-------------------------|---|------------------|--|
| | Reactivity | Doubling time | 10 pcm | |
| (e) | Reactivity | ASM subcritical method (All the configurations) | 4% | Reactivity weights absorbers, exp. device |
| \sim | | Fission U-238/Fission U-235 | 10% | |
| | Spectral indices | Fission Pu-239/Fission U-235 | 6% | Flux |
| | | Fission Np-237/Fission U-235 | 6% | |
| | Neutron Flux | Fission Chambers | 4% | Flux |
| | Temperature coefficient | ASM subcritical method | 5% | Experimental Zone Steps of 20°C (20° to 80°C) |

In-core measurements using **miniature fission chambers** (\emptyset 4, 8 mm), put in the center of the JHR assemblies and in various positions of the driver zone.

•CEA/Cadarache has been developing the miniature fission chambers technology for more than 40 years :

- \emptyset 1.5 mm, 4 mm and \emptyset 8 mm for critical mock-up (CEA, ENEA, JAERI,...) :
 - Various fissile materials :

Th, Np-237, Am-241, U isotopes, Pu isotopes

Ø 4.7 mm for "in-core" neutron flux measurements





Measurements and associated uncertainties (2/4)



| | Parameters | Type of measurements | Uncertainty (2σ) | |
|---|---|---|------------------|--|
| | Conversion ratio | Capture U-238/Total Fission | 4% | Burn up performances |
| œ | Distribution of total fission rates (power) | γ -scanning (All the configurations) | 2% | Power peak Azimuthally, axially, total / plate total / assembly |
| | Axial buckling | Total Fission (spectro γ) | 1.50% | Leakage |

Gamma spectrometry used to measure the fission rates arising in the fuel pins and assemblies:

• the integral γ -scanning technique, used for axial and radial fission rate distributions of the JHR assemblies and UO₂ fuel pins

• *the particular peak check measurement*, used to "renormalize" these integral distributions for the two types of fuel





1 assembly with removable plates

Measurements and associated uncertainties (3/4)



| | Parameters | Type of measurements | Uncertainty (2σ) | |
|-----|--------------------|----------------------|------------------|------|
| | Neutron Flux | Dosimeters | 4% | Flux |
| ran | Axial Distribution | Dosimeters (Au) | 2% | |

Dosimeters

- Determination of the neutron distribution as a function of the energy by irradiating:
 - activation wires between with the fuel removable plates
 - activation disks in the center of the JHR assemblies and in various positions of the driver zone
- Measurement of the gamma emitters in the MADERE platform presented in this conference





Measurements and associated uncertainties (4/4)



| Parameters | Type of measurements | Uncertainty (2σ) | |
|---|---------------------------------|------------------|-----------|
| Total γ heating (Dose γ) | TLD (Al block, Hf, exp. device) | 15% | Γ heating |

Thermoluminescent detectors

- Objective: measure the heating in a material of interest
- \Rightarrow Needs a detector which will integrate the energy of the particles (or the dose)
- Irradiation of thermoluminescent detectors (TLD) => integrated dose in the material
- The absolute dose is obtained after a calibration step of the TLDs (calibrated source): the TLD response is proportional to the dose received
- In order to be representative, the TLD are put in specific boxes, designed for the program (Hafnium and Beryllium boxes)



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100 cm





- AMMON experimental program: carried out in the EOLE facility to provide experimental results for the validation of the Jules Horowitz Reactor HORUS3D neutronics and photonics calculation scheme.
- Design of the program based on a core composed of an experimental central zone with 7 JHR assemblies and an outer driver zone made of standard UO2 PWR pins.
- Several configurations representative of the JHR core in normal operation or accidental situations : successive insertions of a Hafnium control rod, a Beryllium block, an experimental device, a water hole and the voiding of a JHR assembly.
- The flexibility of the EOLE critical facility has allowed the design of this very innovative experimental program:
 - use of *new type of fuel* (JHR assemblies with U_3Si_2 fuel curved plates),
 - modifications of the gamma-scanning devices for measuring curved plates,
 - modification of EOLE to receive a second independent water circuit.
- Large number of measurements planned, using different experimental techniques: characterization of global and local physical parameters => reactivity worth, fine power distributions, spectral indices, gamma heating and temperature coefficient.