



Irradiation effects on 6061-T6 aluminium alloy used for the JHR internal structures

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Reactor block



Reactor aimed features

- Neutron flux $\Phi > 8.10^{14}$ n.cm⁻².s⁻¹ in core center
- $\Phi_{th} / \Phi_{f} \begin{cases} \sim 0.6 \text{ in core center} \\ \sim 13 \text{ in first periphery} \\ \sim 2 \text{ in the vessel} \end{cases}$
- Gamma heating in core center : 16 W/g
- Core slightly pressurized : max 16 bars
- Cooling : light water
- Water velocity : 15 m/s
- Inlet/outlet temperature : 29° C/39° C



Reactor core section

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Fuel element







How to get high neutron flux?

- Choice of Aluminium alloys for the core components :
- Low neutron absorption —> high neutron flux
- Low density —> low gamma heating —> limited temperatures in the core —> moderation/absorption rate / -> neutron flux increase
- Core slightly pressurized :



- Aluminium alloys used for the critical components
- Core vessel : 6061-T6 1%Mg -0,6% Si-Fe-Cu
- Fuel rack : 6061-T6
 high R_m, R_{p0,2}, widely used in USA
 good stability under neutron flux
- Fuel cladding : AIFeNi-O 1%Fe-1%Ni-1%Mg good corrosion resistance up to 250° C low creep strains, used in RHF, FRM2
- Experimental devices : possibly AG3-NET(O)
 2.7% Mg, Mn, Fe → derived from AA-5754 (3% Mg)
 widely used in Europe





➢ 6061-T6 : impact of fabrication process

Tensile and fracture toughness data \longrightarrow great scatter Impact toughness, post-irradiation creep, swelling But no irradiation creep data

AIFeNi-O : lack of mechanical data
 AG3-NET(O) : little data at high fluence
 Tensile data : severe loss of ductility at high fluence
 No fracture toughness, no irradiation-creep and no swelling



6061-T6 : fabrication impact







Swelling of aluminium alloys



K.Farrell, 1995 ORNL/TM-13049



- Qualification program of aluminium
 - 2000-2005 : definition phase
 - Development of fabrication and welding processes
 to assess the industrial feasibility
 - Since 2006 : qualification of fabrication processes
 - Definition of receipt criteria
 - Qualification of relevant processes : forging, welding
 validate different methods, link process/properties
 - Implementation of two irradiation programs in OSIRIS

validate mechanical properties during design and operation



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Forging process



Core vessel demonstrators



Microstructures obtained on Servotest samples in different conditions (T, strain rate)





Welding process

✓ Use of a Welding Procedure
 Qualification for aluminium

✓ Several welding and repairing processes tested : electron beam, arc welding, with or out filling metal

✓ Use of European standards EN 15614-11 and 15614-2

 ✓ welding control : Soundness control, destructive tests, metallographies

Electron beam soldering with filling metal





RAJAH irradiation in Osiris

- Started in April 2008
- Alloys irradiated : 6061-T6 , AG3-NET(O) and 6061-T6 welded samples
- 3 sample types : tensile, impact and fracture toughness
- Neutron spectrum : $5 < \Phi_{th} / \Phi_{fast} < 10$ (conservative in regard to JHR vessel)
- Irradiation temperature : 40° C < T < 50° C
- Thermal neutron flux in mid-plane (E=0.0254 eV): 2.2x10¹⁴ n.cm⁻².s⁻¹
- 15 baskets containing 154 samples
- To get homogenous fluences : permutation of the baskets every 4 cycles, and device turned 180° at each cycle
- To get samples at higher fluence : 3 central baskets remain in the mid-plane





RAJAH device





Irradiation place in Osiris





RAJAH irradiation targets

First phase, started in April 2008 :

- Forged 6061-T6 : different fabrication processes
 impact on the mechanical properties evolution up to 1.5x10²² n_{th}.cm⁻²
- Forged AG3-NET(O) : get data up to 1.5x10²² n_{th}.cm⁻²
- Welded 6061-T6 samples : different welding processes

show that the vessel welding joints are not degraded by the irradiation at a fluence representative of 10 years operation

Second phase, to start in March 2010 :

• 6061-T6 fabricated with the process chosen for the vessel :

Irradiation up to ~ $1.5 \times 10^{22} n_{th} \cdot cm^{-2}$; 3 test temperatures : $20^{\circ} C$,

75° C (JHR vessel) and 125° C (rack T excursions)

validation of the vessel fabrication process





RAJAH third phase, scheduled to start at the middle of 2011

- JHR vessel qualification irradiation + presurveillance program
- Thermal fluence received in Osiris (24 cycles) : max 10²² n.cm⁻²

(equivalent of ~1 year operation of the JHR vessel)

- A few samples tested : vessel qualification
- Most samples transferred into the JHR reactor for a vessel surveillance program



FLOREAL irradiation in Osiris



- Started in December 2006
- Aims to assess the in-flux relaxation behaviour of aluminium alloys and to predict their irradiation creep behaviour
- Alloys irradiated : 6061-T6, AG3-NET(O) and AIFeNi(O)
- Sample types : strip shaped ; 3-point-bending relaxation technique
- Neutron spectrum : $\Phi_{th} / \Phi_{fast} \approx 2$ similar to that of JHR core vessel
- Fast neutron flux : $\Phi_{fast} = 2x10^{14} \text{ n.cm}^2.\text{s}^{-1}$ (E>1MeV)
- Irradiation temperature : 40° C < T < 45° C
- 7 baskets containing 56 samples
- 2 types of bended positions to set 2 different stresses : 45% and 75% of yield strength
- Bended samples periodically unloaded and measured in hot cells with a laser beam —> residual deflection —> relaxation strain rate





Bended sample

Unbended sample



Irradiation place in Osiris







FLOREAL irradiation

- First part, from December 2006 to June 2009 :
- Aimed to measure the relaxation behaviour of as-fabricated alloys
- 8 irradiation periods (3 days up to 3 cycles)
- Fast fluence received : 5.25x10²¹ n.cm⁻² (E>1 MeV)
- Second part, from October 2009 to the end of 2012
- Aims to measure the relaxation behaviour of samples pre-irradiated in unbended positions to $\Phi_{th} = 1.3 \times 10^{22} \text{ n.cm}^{-2} \text{.s}^{-1}$
- Some of the bended samples (first part) stay in bended positions to increase their fluence up to ~ 10²² n_{fast}.cm⁻²
- 8 irradiation periods (3 days up to 3 cycles)
- Fast fluence received : 5.25x10²¹ n.cm⁻² (E>1 MeV)